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 An ideal introduction to the fundamentals of nuclear science and engineering Presents the basic nuclear science needed to

AN MCNP PRIMER - Mechanical and Nuclear Engineering

J Kenneth Shultis and Richard E Faw The MCNP Code, developed and maintained by Los Alamos National Laboratory, is the internationally recognized code for analyzing the transport of neutrons and gamma rays (hence NP for neutral particles) by the Monte Carlo method (hence MC) The code deals with transport of neu-

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Introduction to nuclear engineering; Global and national energy requirements; Radioactivity; Atomic models; Fission and f References: Textbooks: Available Electronic Books 1 Manson Benedict, NUCLEAR CHEMICAL ENGINEERING, 2nd edition, 1981 2 Kenneth D Kok, Nuclear Engineering Handbook, 2009 3

HALF-SPACE GENERAL MULTIGROUP TRANSPORT THEORY*

Journal of Nuclear EnergyVol 23 pp 477 to 493 Pergamon Press 1969 Printed in Northern Ireland HALF-SPACE GENERAL MULTIGROUP TRANSPORT THEORY* S PAHOR~ and J KENNETH SHULTIS\$ Department of Nuclear Engineering, The University of Michigan,

FUNDAMENTALS OF NUCLEAR SCIENCE AND ENGINEERING

nuclear engineering option into a much larger mechanical engineering program at Kansas State University This book was designed to serve both as an introduction to the students in the nuclear engineering option and as a text for other engineering students who want to obtain an overview of

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Department of Mechanical and Nuclear Engineering College of Engineering KANSAS STATE UNIVERSITY Manhattan, Kansas 2009 Approved by:
Major Professor Dr Kenneth Shultis ii Abstract A program which calculates the radiation dosage to a predetermined set of components

Approximate Equation to Correct Point Source Assumption

[7] J Kenneth Shultis and Richard E Faw, Radiation Shielding, American Nuclear Society, Illinois, USA, p164-, 2000 [8] X-5 Monte Carlo Team,
MCNP-A General Monte Carlo N-